

$^{241}\text{Am}(n,\gamma)$ cross section in the neutron energy region between 0.02 eV and 300 keV

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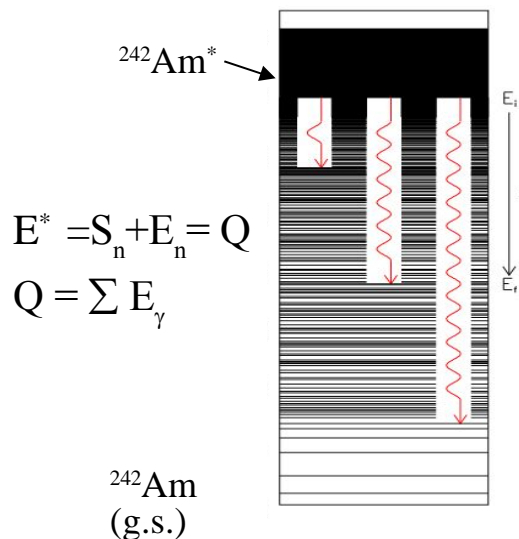
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The Detector for Advanced Neutron Capture Experiments (DANCE)

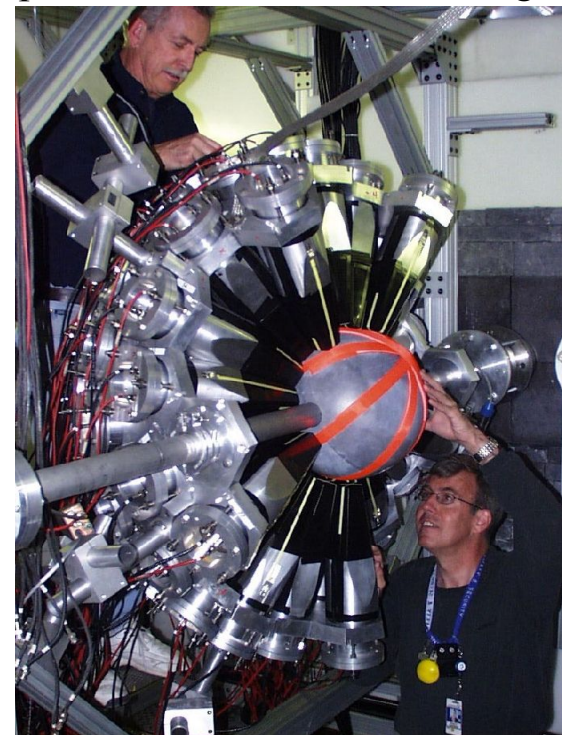


- Neutron capture process: $n + ^{241}\text{Am} \rightarrow ^{242}\text{Am}^*$, cascade of γ -rays is emitted from excited compound nucleus to reach to g.s. of ^{242}Am
- Excitation energy of $^{242}\text{Am}^*$

$$E^* = S_n + E_n = Q = \sum E_{\gamma_i}$$
- By γ -ray detection the capture process can be identified using total absorbed energy

DANCE

- Located at the Lujan Center at the Los Alamos Neutron Science Center on flight path 14
- 160 BaF_2 crystals in 4π geometry
- ^6LiH ball placed around the target to attenuate scattered neutrons
- Designed to run with radioactive targets (up to 1 Curie)



Data Acquisition

- 324 channels of Acqiris DC265 digitizers 500 Msamples/s
- 14 frontend computers
- Raw Data:
 - Typical BaF₂ signal consist from slow and fast component
 - Software CFD identifies the signal
 - The fast component: 32 points of a waveform (64 ns)
 - The slow component: 5 x 100-point sections which are integrated (5x200 ns)
 - Reduced waveform is then stored to disk with the timing information
- Off-line Analysis:
 - In the software coincidence window the events are identified
 - For each event we extract (after proper calibrations)
 - Time of the leading edge - E_n
 - Slow and fast component integral - E_γ
 - Multiplicity of crystals and **clusters**
- DAQ - *J.M. Wouters et al., IEEE Transactions on Nucl. Sci., Vol. 53, No 3., (2006) 880*

Parallel Plate Avalanche Counter (PPAC)

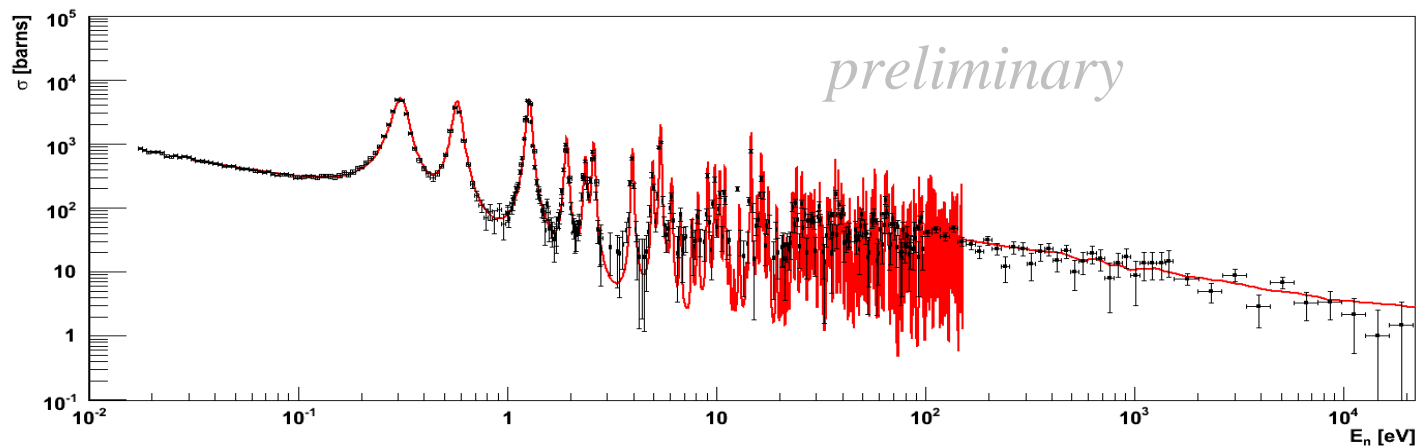
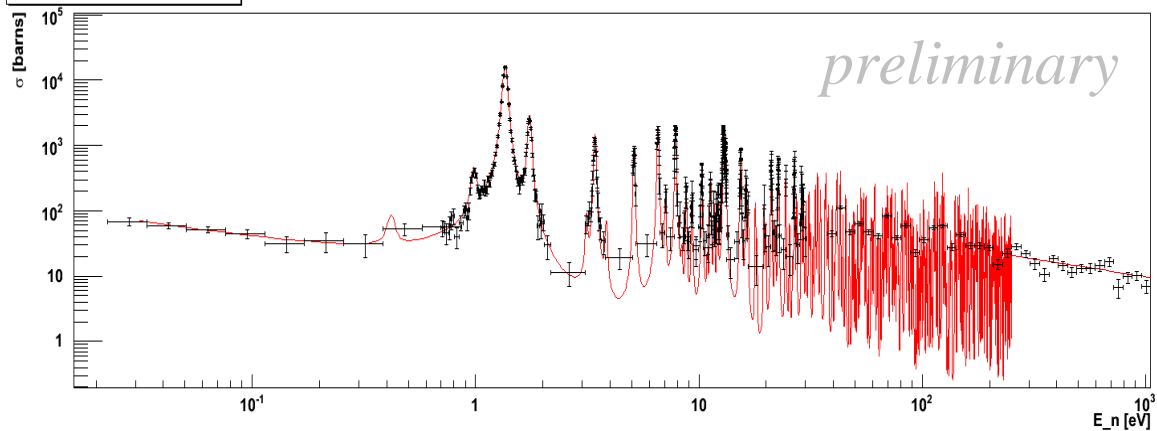
- Designed to surround the actinide target to help separate competing (n,f) and (n,γ) processes
- The goal is to measure $(n,f)/(n,\gamma)$ and make improved (n,γ) measurements
- PPAC is used for fission tagging (no info on mass and energy of the fission fragments)
- Working conditions:
 - Two anodes at +250V
 - Cathode “target” at -250V with the sample
 - Isobutane gas flowing at 8 ccm
 - Pressure 6 torr



PPAC is customized to fit in the beampipe

2005 - Results

$^{241,243}\text{Am}$ cross section (n, γ)

Am241 (n, γ) Cross Section ^{243}Am (n, γ) Cross Section

- PPAC untagged events analyzed
- Larger uncertainties at small σ
- Comparison with ENDF VII (red lines)
- (n,f) contamination is less 0.1 %

2006 - $^{241}\text{Am}(n,\gamma)$ measurement

- no need for PPAC, cuts on multiplicity and E_{tot} will reduce fission contribution to tenths of a percent
- August 2006 - 17 days of continuous measurement, included data taking in segmented mode 14ms, 2ms (0.02-36eV) and continuous mode above 36 eV, with the sample and blank targets

Experimental quantities to determine

$$\sigma = \left(\frac{A}{N_A \rho_s S} \right) \frac{N_{n,\gamma}}{F \varepsilon_{cut} \varepsilon_{tot}}$$

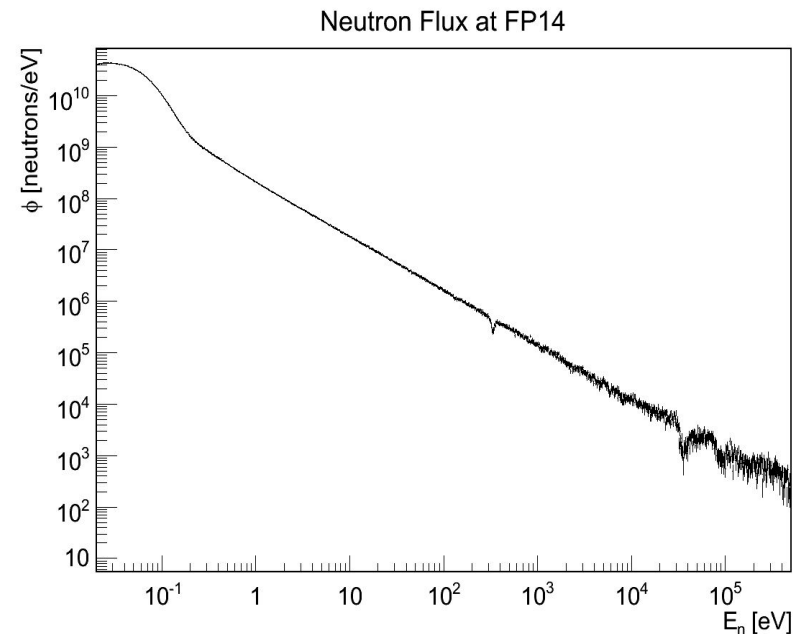
- Mass of the target material $M (\rho_s S)$
- Neutron flux F per cm², per second, per eV on target
- Total efficiency ε_{tot} of detecting at least one γ -ray from the de-excitation cascade that follows neutron capture
- Efficiency of detecting the cascade ε_{cut} after applying the cuts on experimental data, namely γ -ray multiplicity and total energy of γ -rays
- Flux of γ -rays corresponding to the neutron capture cascade $N_{n,\gamma}$, after the background subtraction

²⁴¹Am target

- electroplated on two Ti 2.5 um foils sandwiched back-to-back
- covered from both sides with thin aluminized mylar foils
- very pure 99.9% material
- diameter ¼ of inch (6.35 mm)
- total mass from gamma-counting 203(1) ug
- total mass from alpha-counting 219(1) ug
- discrepancy due to the loss of material after alpha counting

Neutron Flux Measurements

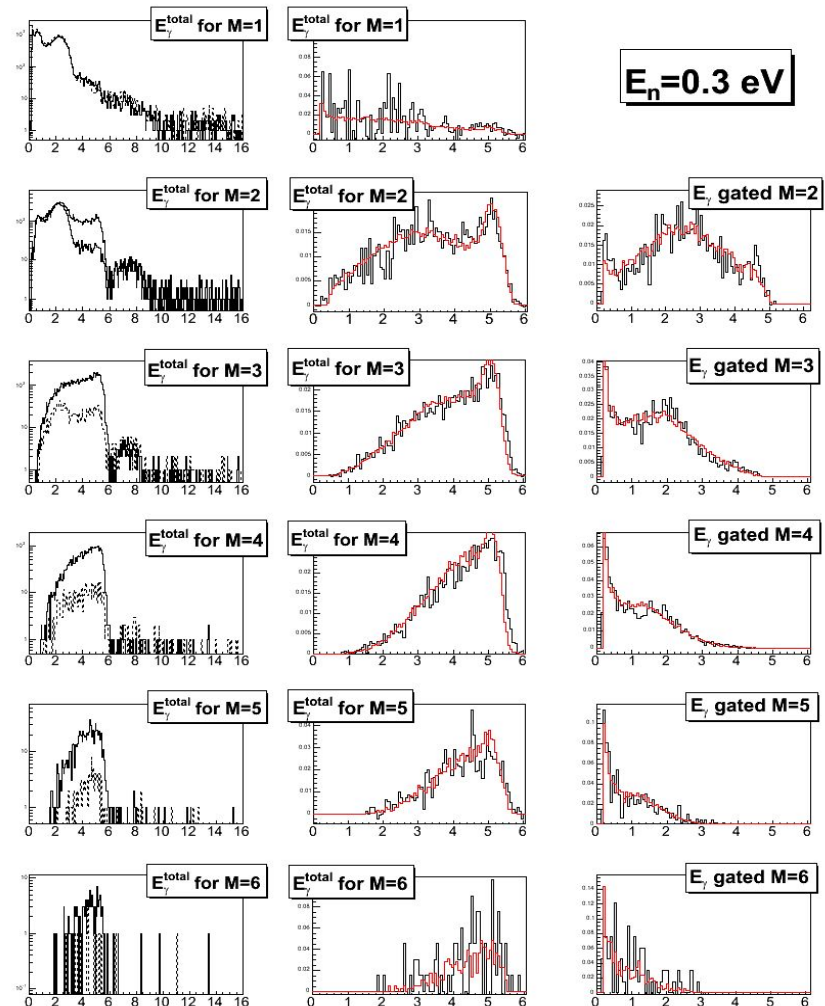
- FP14 uses three neutron beam monitors
 - 1) foil with enriched ${}^6\text{Li}$. Products of the break-up are detected in Si detector position perpendicular to beam, 2 cm away from the foil. The foil is at 45 degrees.
 - 2) BF3 monitor – ${}^{10}\text{B}(n,\alpha){}^7\text{Li}$
 - 3) U monitor – fission fragments ${}^{235}\text{U}$
- three monitors are triggered with the common T_0 trigger as DANCE
- concurrent measurement
- monitors can cover 0-14 ms after T_0



Typical cumulative flux measured with BF3 monitor

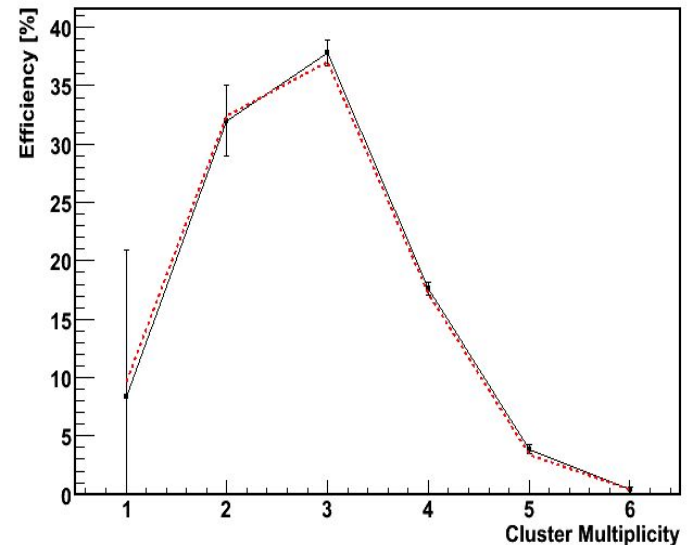
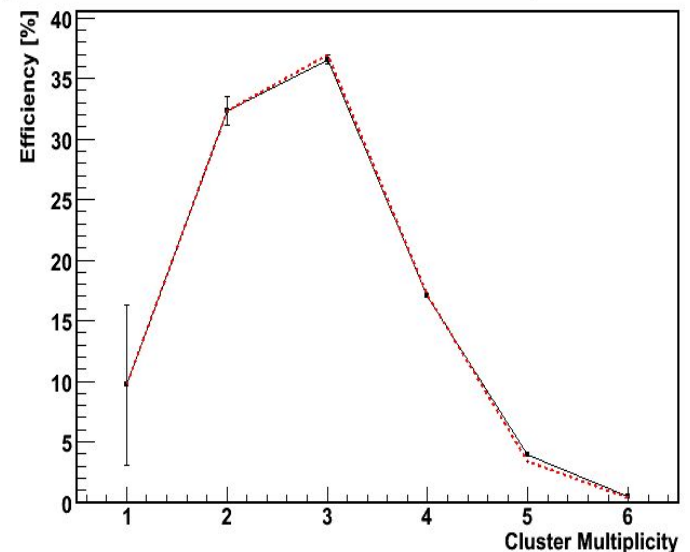
^{241}Am γ -cascade detection efficiency

- at resonances the multiplicity distribution can be analyzed
- easy background subtraction
- red lines is DICEBOX-Geant4 calculation assuming average 2.5 spin state
- very good agreement in gamma-ray spectra between experiment and calculation
- withing experimental uncertainties one cannot distinguish spectra from two different resonances !



Cascade efficiency

- multiplicity distribution identical for both resonances
- red lines is DICEBOX-Geant4 calculation assuming average 2.5 spin state
- very good agreement in gamma-ray spectra between experiment and calculation
- withing experimental uncertainties one cannot distinguish spectra from two different resonances !
- the consequence is very important for cross sections analysis, we can apply E_{tot} and M cuts for any neutron energy without the need to make correction for different cascade efficiencies due to different spin state
- Signal-to-noise was found to be best for following cuts: **$M=4$ and $E_{tot}=3.675-6$ MeV**, with the total efficiency **0.125(1)**.

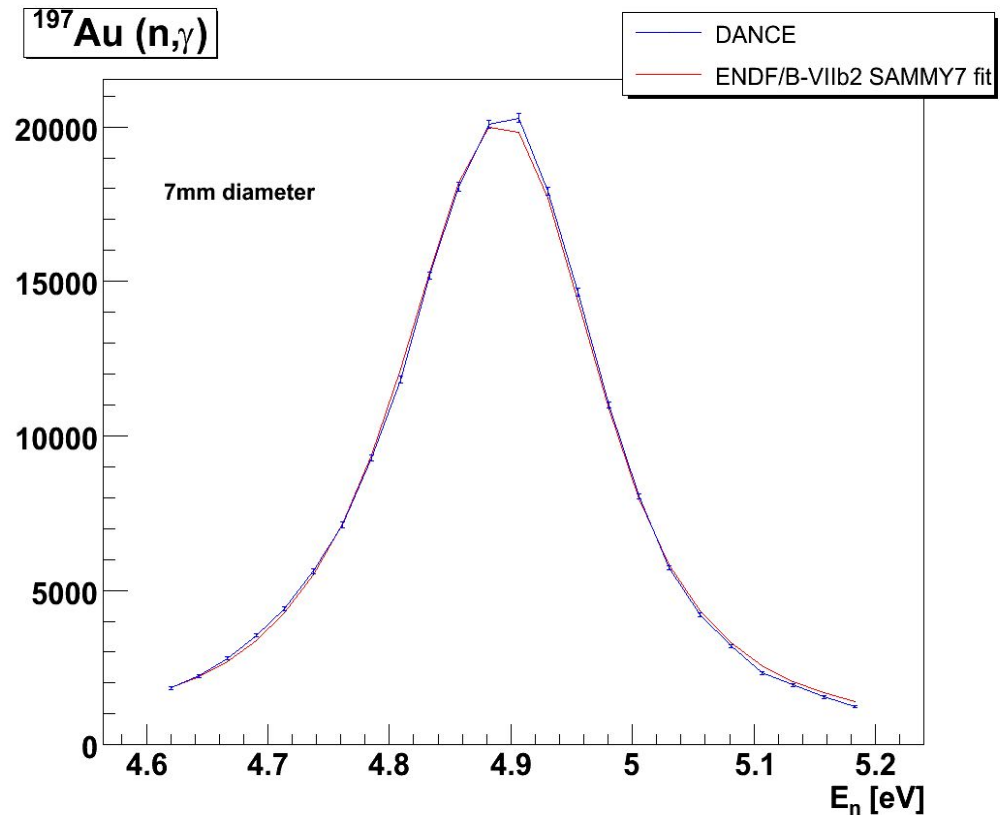
Efficiency deduced from 0.3eV resonance in ^{241}Am (n,γ)Efficiency deduced from 1.27eV resonance in ^{241}Am (n,γ)

Monitor Flux Normalization

- The idea is to determine the flux at target position using ^{197}Au target with known thickness and the same diameter
- Assuming the shape of flux spectrum doesn't change from experiment to experiment (well confirmed)
- we had 7mm diameter target available of 487.5 nm thickness
- flux measured at the monitor position $F_{\text{target}} = a f_{\text{monitor}}$
- using
$$\sigma = \left(N_A \frac{\rho_s}{A} \right)^{-1} \frac{N}{FS} = \left(N_A \frac{\rho_s}{A} \right)^{-1} \frac{N}{afS}$$
- the fit was performed with SAMMY on 4.9 eV resonance with EDNF7 parameters

Monitor Flux Normalization using Au targets

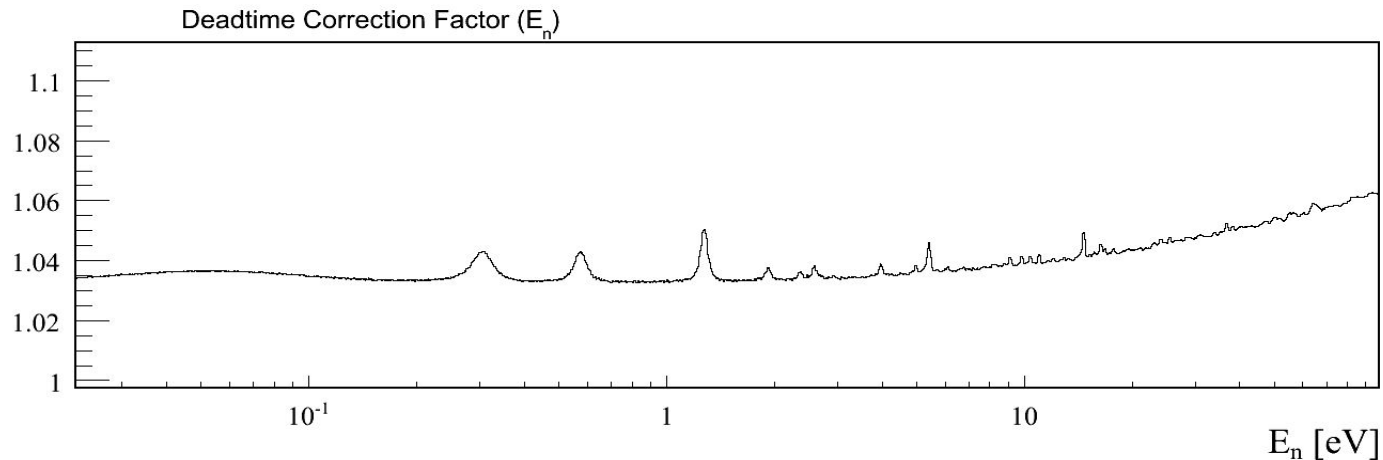
- Flux measured with BF_3 monitor during $\text{Au}(n,\gamma)$ measurement
- self shielding correction applied on the data
- 7mm diameter - 478.5 nm thickness
 - $a = 0.866$
- 4mm diameter - 492.5 nm thickness
 - $a = 0.887$
- to reduce uncertainties we will remeasure flux profile at target position



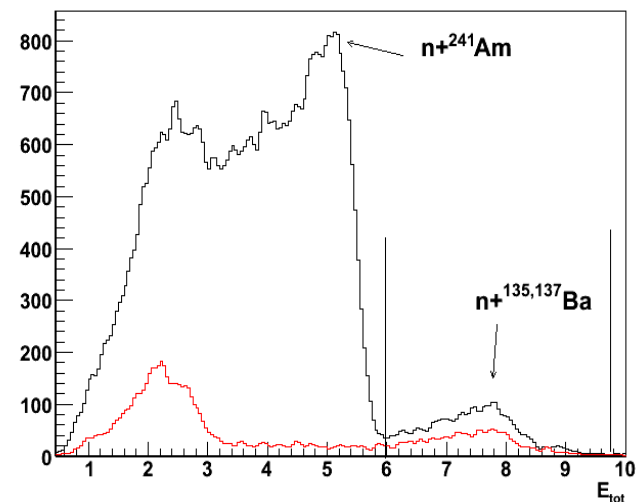
Summary of important variables

- Mass of ^{241}Am : $219(1) \mu\text{g}$
- Best signal-to-noise ratio: $M=4, E_{\text{tot}}=3.675\text{-}6 \text{ MeV}$
- Total cascade efficiency: $\varepsilon_{\text{tot}}\varepsilon_{\text{cut}} = 0.125(1)$
- Flux normalization factor: $a = 0.870(1)$

N(n, γ) spectrum

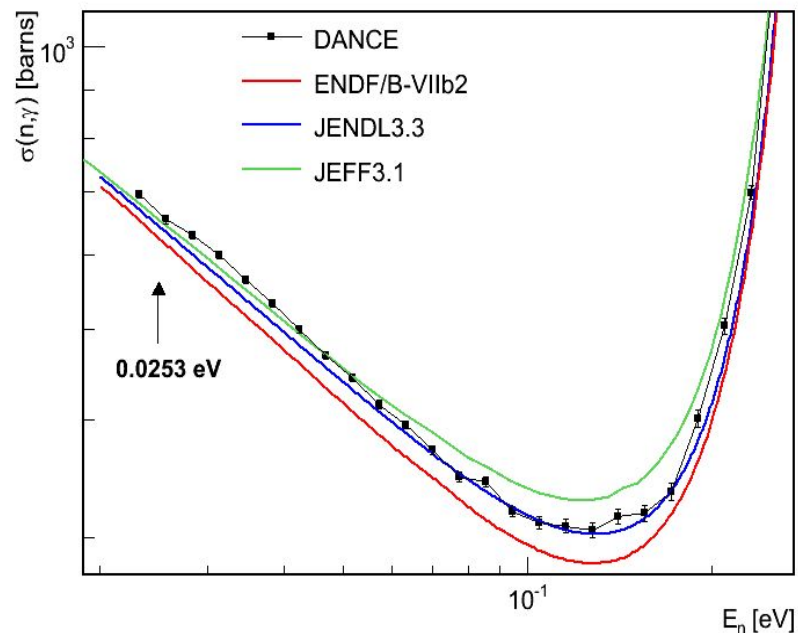
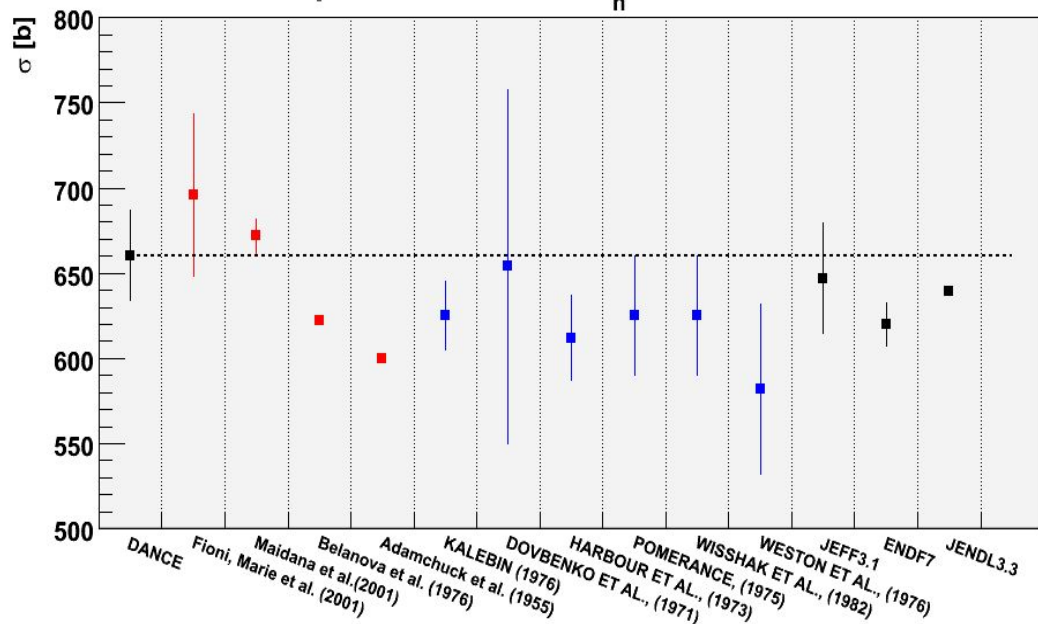


- In segmented mode we apply dead time correction
- Best signal-to-noise ratio: $M=4$,
 $E_{\text{tot}}=3.675-6$ MeV
- Background is subtracted using the blank runs and normalizing on Ba capture peaks in every bin

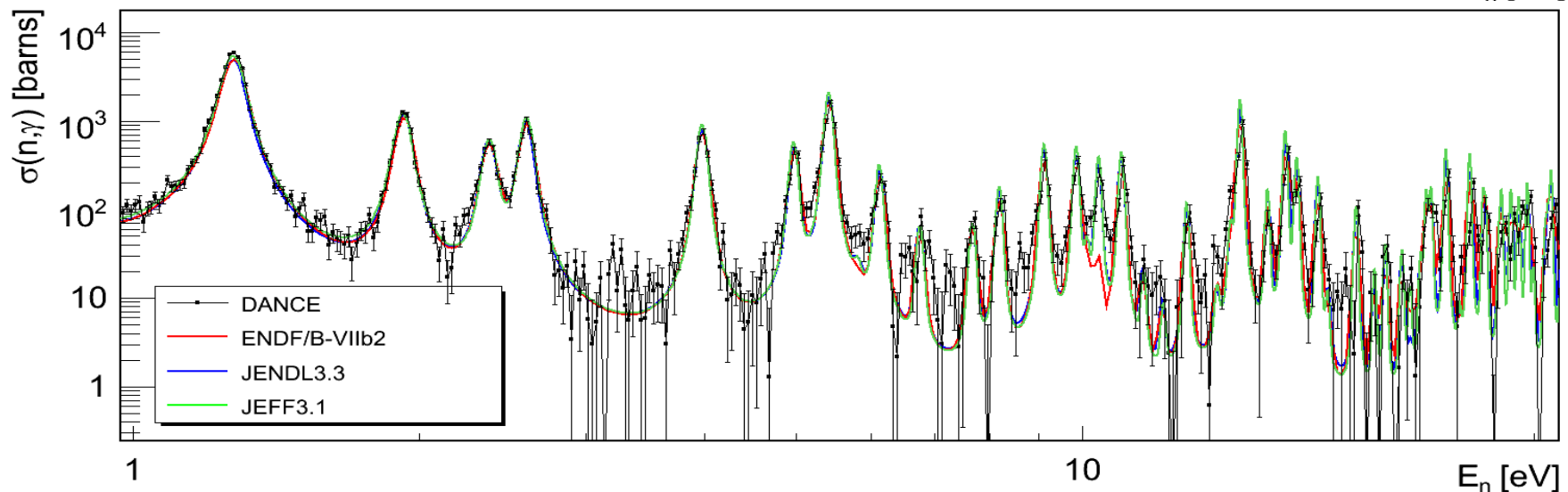
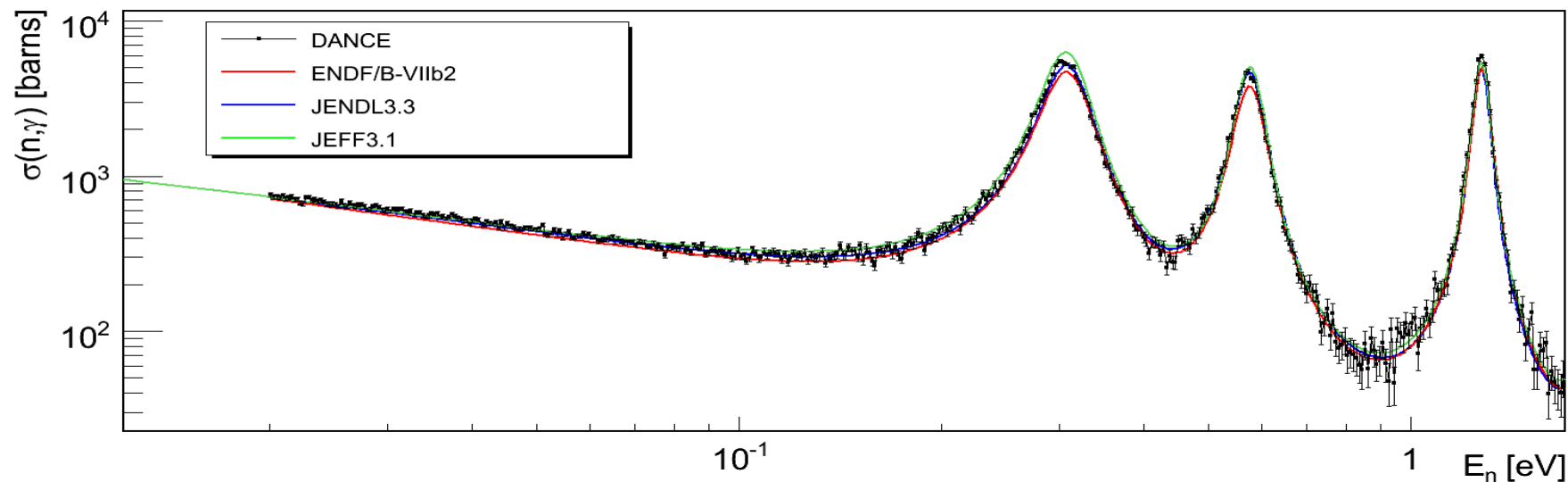


Cross section at thermal

Experimental status $E_n = 0.0253 \text{ eV}$

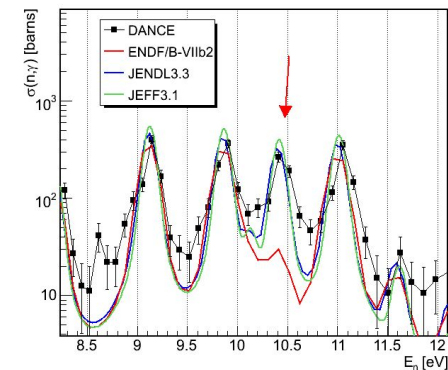
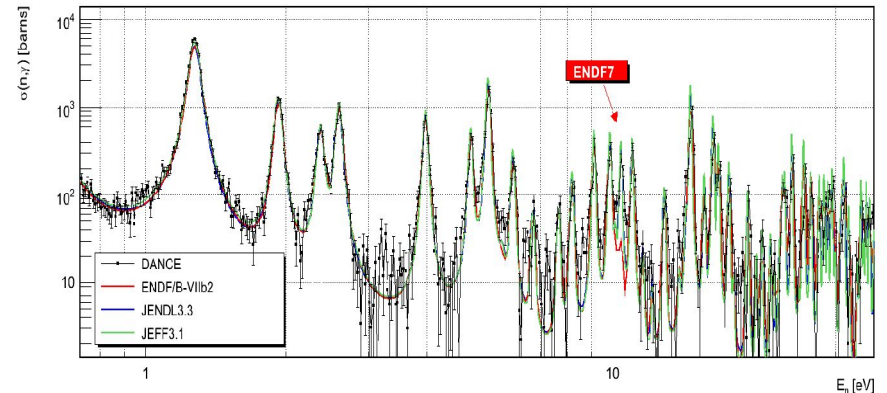
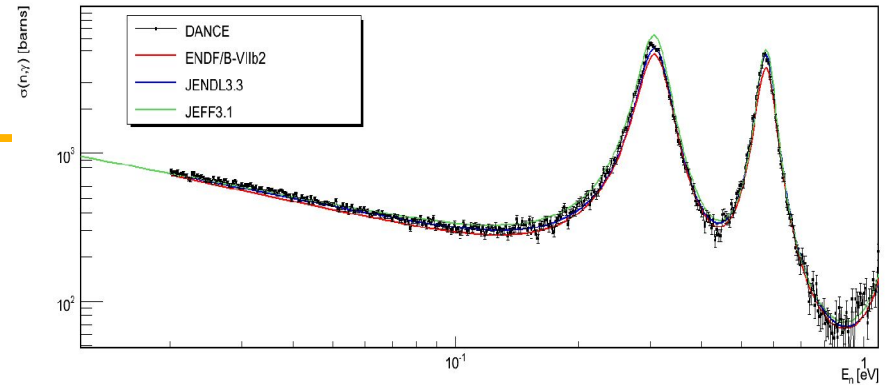


Cross section 0.02-36 eV (bins 1%)



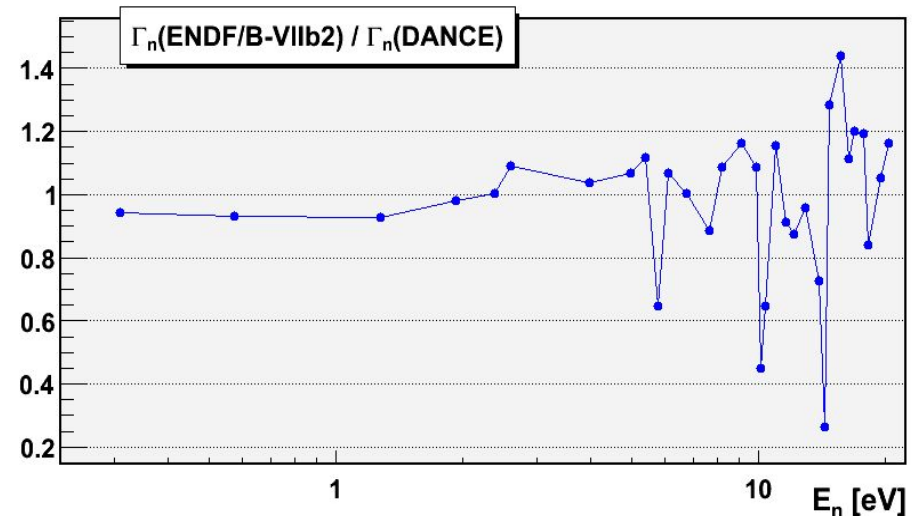
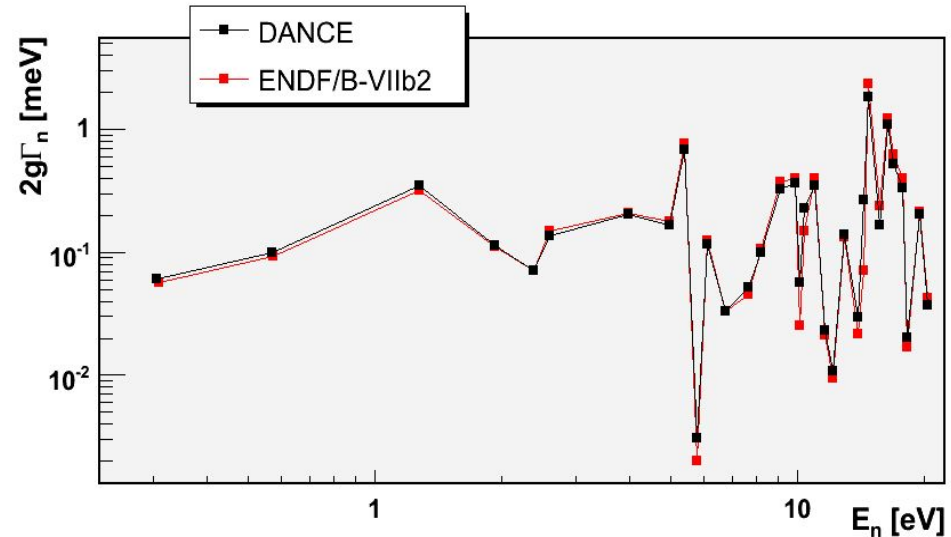
Cross section 0.02-36 eV

- At first two resonances our data are closer to JEFF and JENDL evaluations than ENDF
- self shielding correction was not applied yet on the data, and is planned at the last stage when precise SAMMY fit will be performed
- resonance at 10.4 eV is underestimated in ENDF evaluation (our analysis excludes fission contribution to a high degree)
- statistical uncertainties are decided manly with dE/E choice
- at thermal (0.02-0.1eV) 4% systematic uncertainty should be considered, stemming from the inconsistency between our beam monitors

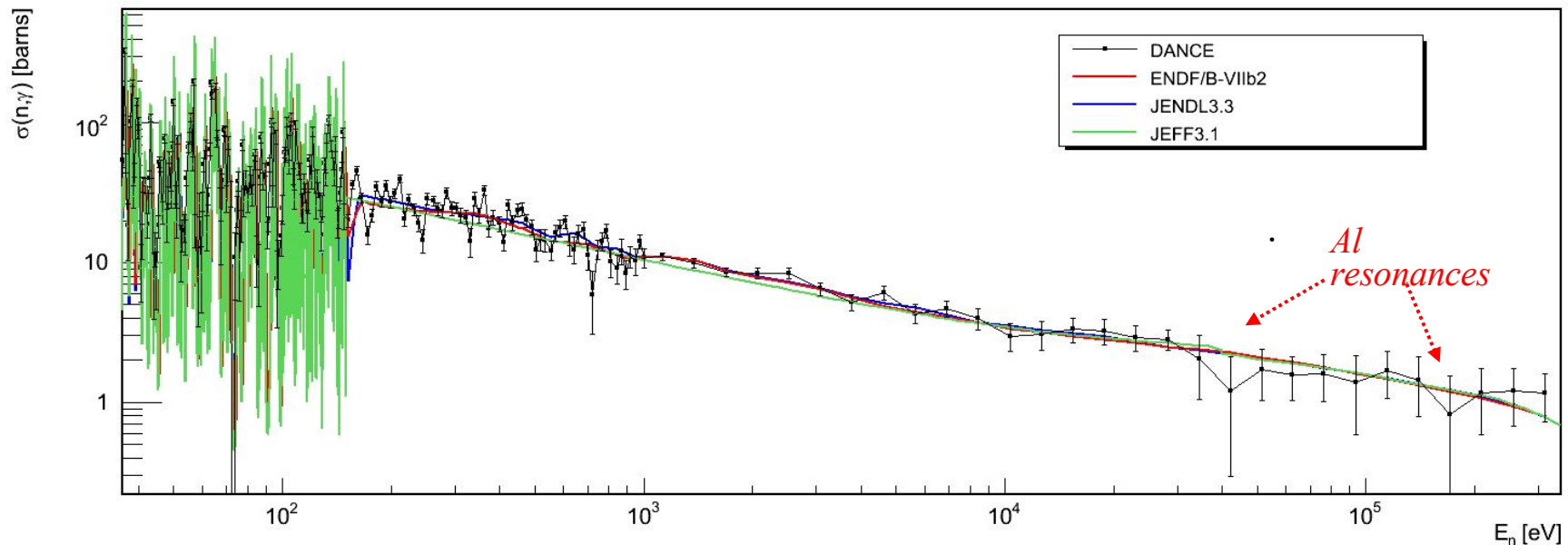


Resonance Parameters

- comparison to ENDF/B-VIIb2
- SAMMY7 was used to fit resonances below 20 eV
- self shielding + multiple scattering effects included in the fitting procedure
- neutron widths as a fitting parameter
- gamma width kept from ENDF
- $2g\Gamma_n$ are compared in the upper panel
- lower panel shows the ratio of Γ_n ENDF/DANCE

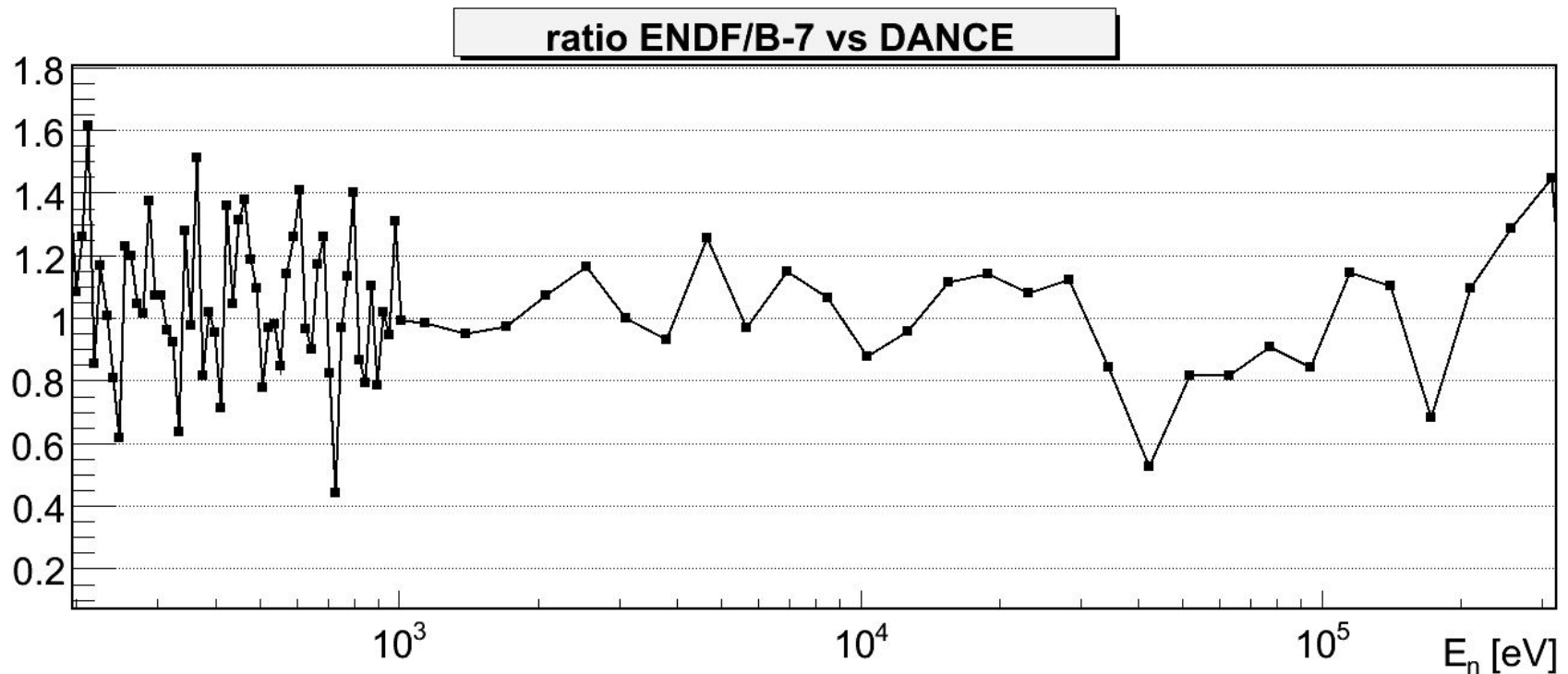


Cross section 36 eV – 300 keV



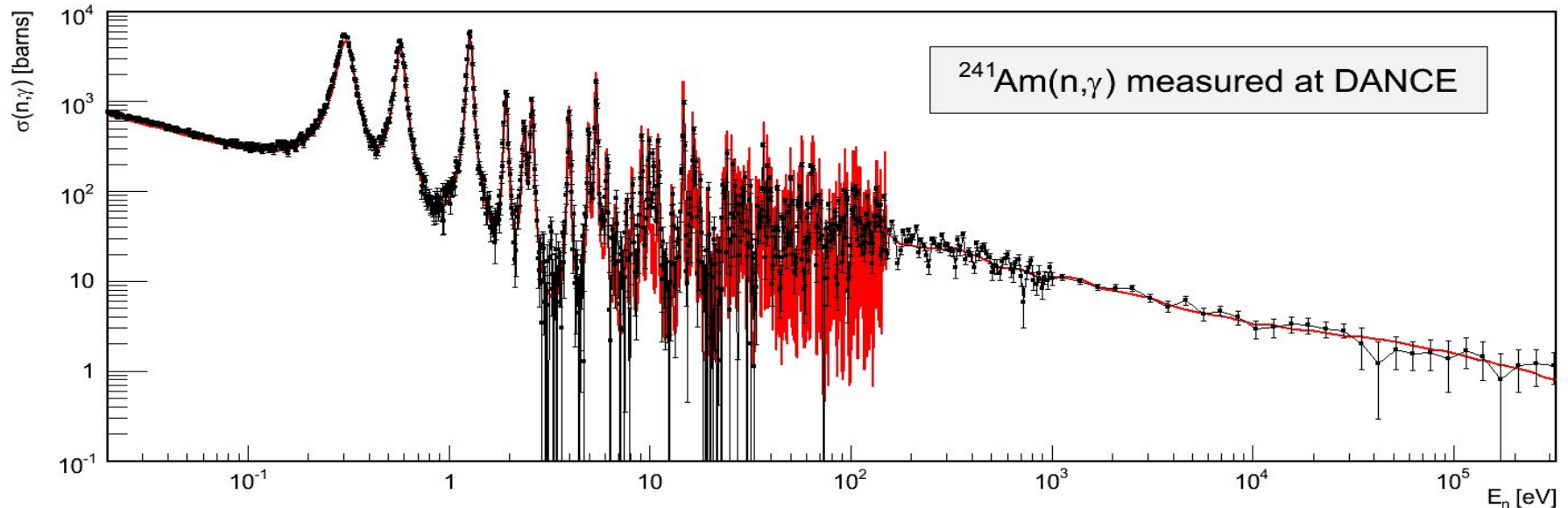
- Our data agree well with ENDF and JENDL evaluations. In 1 keV region the trend follows these two
- The data points at 30 keV and 180 keV are effected by the aluminum in the flight path and should not be considered

Cross section 36 eV – 300 keV



- Our data agree well with ENDF and JENDL evaluations. In 1 keV region the trend follows these two
- The data points at 30 keV and 180 keV are effected by the aluminum in the flight path and should not be considered

Cross section 0.02 eV – 320 keV



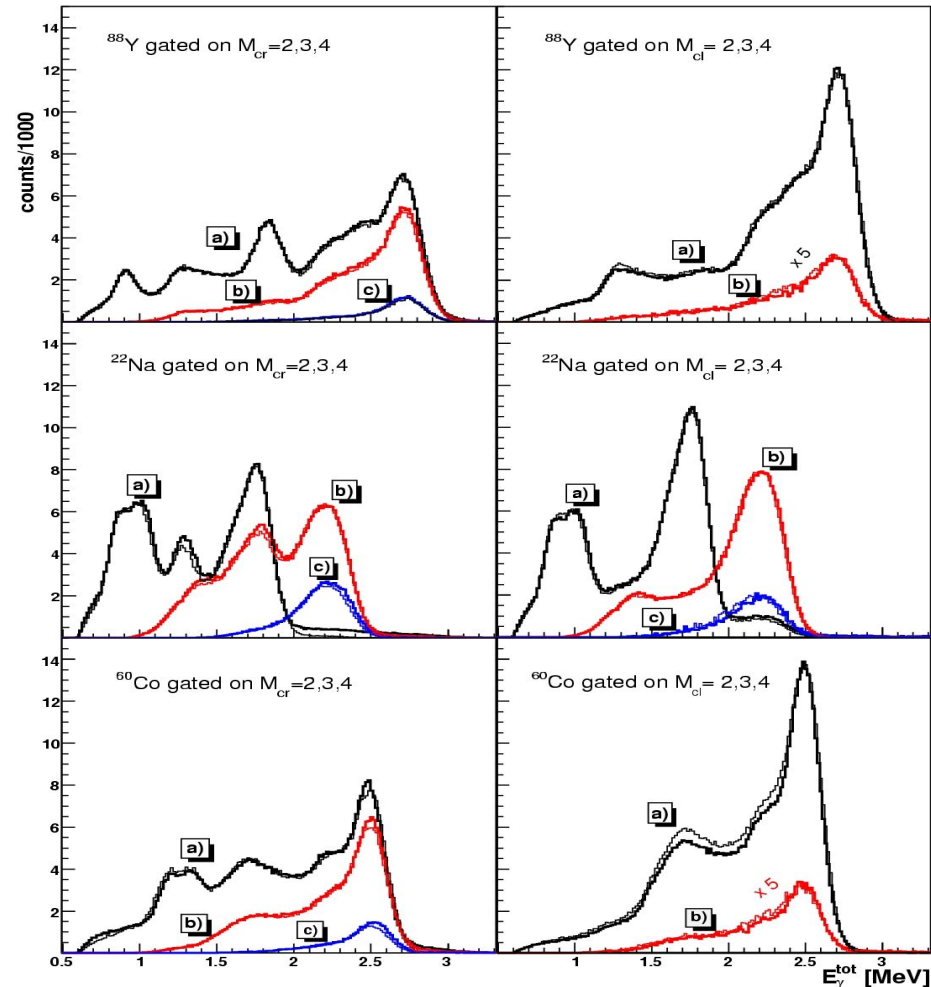
- DANCE measurement results compared to ENDF7b (red line) in the full range of neutron energies
- no weighing is assumed – results represent average cross sections in E_n bins, the points are in the center of bins
- binning convention $\Delta E/E$ 1% (0.02-150 eV), 3% (150 eV -1 keV) and 20% above 1 keV
- uncertainties: 4% thermal, 6% 1KeV, 18 % 10 keV

Major Achievements

- It is possible to perform absolute cross section measurements at DANCE in the span of 7 orders of magnitude of neutron energy
- We found reliable way how to normalize the flux at target position using Au targets
- Method to determine the cascade detection total efficiency was developed
- Computer simulations (Geant4, DICEBOX) were performed with good agreements
- Software treatment of the data, both in on-line data collection and off-line analysis has been improved

DANCE-GEANT4 code – Calibration sources

- ^{88}Y , ^{22}Na , ^{60}Co
- Total γ -ray energy spectra gated on cluster and crystal multiplicity
 - Experiment (thick lines)
 - GEANT4 (thin lines)
 - M=2 (black)
 - M=3 (red)
 - M=4 (blue)
- Overall good agreement between the simulation and experiment (3%)



Cascade efficiency

- relative efficiency for different cuts on E_{tot} compared to DICEBOX-Geant4 calculations

